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AUGUST - 1989 EM-4180110-179

ROCKÝ FLATS



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PLANT

MONTHLY
ENVIRONMENTAL
MONITORING
REPORT

ENVIRONMENTAL MANAGEMENT

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Confirmed Unclassified, Not UCNI/Not OUO

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AUGUST 1989 ENVIRONMENTAL MONITORING REPORT ROCKY FLATS PLANT

This report summarizes the effluent and environmental monitoring programs at the Rocky Flats Plant for the month of August 1989. The data presented herein are the best information available to the Rocky Flats Plant at this time. Should subsequent analyses indicate that any data presented herein are inaccurate or misleading appropriate revisions will be issued promptly.

Included in the report are monitoring results for radioactive and nonradioactive airborne effluents continuously sampled from Plant buildings, Tables I and II. Tables III through V summarize environmental monitoring data from the Rocky Flats Plant ambient air sampling network. This network is comprised of continuously operating air samplers located on plantsite, around the Plant boundary, and in neighboring communities.

Water sampling results for radioactive constituents are given in Tables VI through VIII. Results are summarized for Plant surface water control ponds, for nearby drinking water reservoirs, and for tap water for neighboring communities. Nitrate monitoring for Great Western Reservoir and Standley Lake, the two drinking water reservoirs which can receive surface water discharges from the Plant, are summarized in Table IX.

The Environmental Protection Agency (EPA) has issued to the Plant a National Pollutant Discharge Elimination System (NPDES) permit for control of surface water discharges. Water sampling results associated with the NPDES permit, as well as applicable discharge limitations imposed by that permit, are reported in Table X. Analytical results for nonradioactive parameters in water at the Walnut Creek at Indiana Street location are summarized in Table XI. Daily flow data for surface water from the two Plant drainage systems are given in Tables XI, XII, and XIII.

The data provided in this report are provided as a matter of comity and should not be construed as an application for a permit or license, or in support of such an application. Approval of the Department of Energy should be obtained prior to publication of any data contained within this report.

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Directed
by J.A. Nesheim DOE M47).3-1

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Table I. 1989 Plutonium and Uranium Airborne Effluent Data

	<u>Plu</u> (07/20/89 -	tonium 08/21/89 - AUG)		ranıum 08/22/89 - AUG)
Manth	Release	08/21/89 - AUG) Max (pC1/m ³)	(07/21/89 - Release	CMay
<u>Month</u>	<u>(uC1)</u>	(pc1/m²)	<u>(uC1)</u>	(pC1/m ³)
CY 1988	15.07	0.023 ± 0.0052	11.28	0.009 ± 0.0009
January	0.33	0.005 ± 0.0005	0.15	0.000 ± 0.0001
February	0.15	0.001 ± 0.0001	0.20	0.001 ± 0.0002
March	0.07	0.001 ± 0.0001	0.04	0.002 ± 0.0002
April	0.28	0.001 ± 0.0001	0.04	0.001 ± 0 0001
May	0.18	0.001 ± 0.0001	-0.03	0.001 ± 0.0001
June	0.06	0.001 ± 0.0001	0.06	0.001 ± 0.0002
July	0.18	0.001 ± 0.0002	0.15	0.001 ± 0.0002
August	0.07	0.001 ± 0.0002	0.02*	0.000 ± 0.0002*
September				
October				
November				
December				
Year to Date	1.31	0.005 ± 0.0005	0.63*	0.002 ± 0.0002*

^{*} One incomplete analysis.

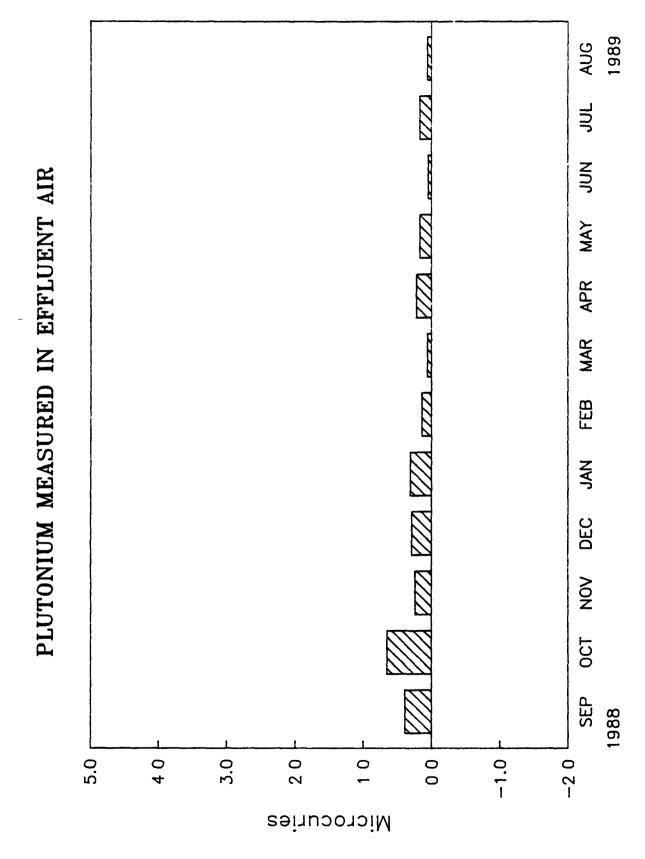
NOTE: The plutonium, uranium, americium, and beryllium measured concentrations in this report include values that are less than the corresponding calculated minimum detectable concentrations (MDC's). In some cases, the values are less than zero. This method of reporting began in January 1981 These negative values result when the measured value for the laboratory reagent blank is subtracted from an analytical result which was measured as a smaller value than the reagent blank. This may happen when measuring concentrations which are very close to zero

Table II. 1989 Tritium and Beryllium Airborne Effluent Data

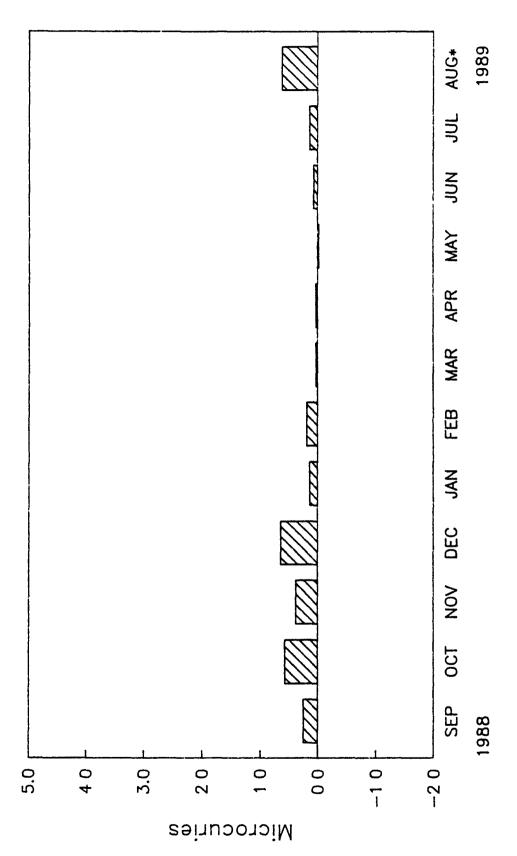
		ntium	Beryllıum		
				08/22/89 - AUG)	
Month	Release (C1)	CMax (pC1/m3)	Release (grams)	CMax (ug/m3)	
CY 1988	0.014	417 ± 250	0.1322	0 00041	
January	0.001	97 ± 145	0.0285	0.00033	
February	0.002	166 ± 120	-0.0392	-0.00005	
March	0.007	389 ± 220	-0.0025	0.00000	
April	0.152	14000 ± 320	-0.0031	0.00017	
May	0.003	65 ± 35	0.0024	0.00004	
June	0.001	99 ± 10	0.0525*	0.00025	
July	0.001	108 ± 13	0.1727*	0.00106	
August	0.006	2735 ± 34	0.1343*	0.00100	
September					
October					
November					
December					
Year to Date	0.167	14000 ± 320	0.2113	0.00106	

^{*} No Blank Correction

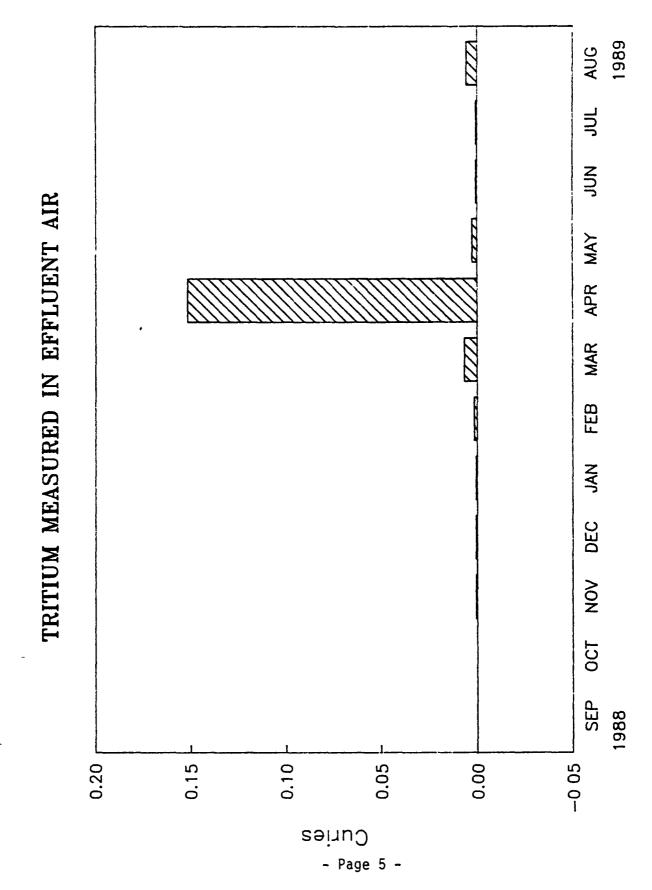
NOTE: Beryllium measured at the remaining 44 locations was below the screening level of 0.1 gram per month.



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BERYLLIUM MEASURED IN EFFLUENT AIR

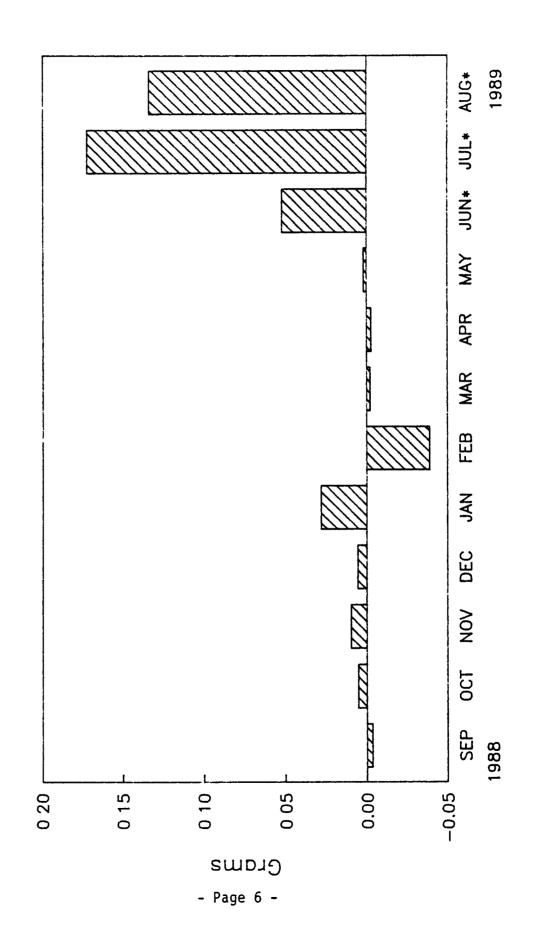


Table III.

Plutonium Concentration in Ambient Air for Selected Onsite Samplers

AUGUST 1989 (07/25/89 - 08/22/89)

Location	<u>N</u>	Volume <u>(m³)</u>	Avg. Pu Conc. <u>(pCı/m³)</u>	+/- Error <u>(pCi/m³)</u>
S-05	*	0		
S-06	2	33000	0.000448	0.000097
S-07	2	27000	0.000453	0.000108
S-08	2	33000	0 000752	0.000170
S-09	2	29000	0.000459	0.000103

NOTE: The total long-lived alpha activities of the remaining onsite ambient air sampler filters were below 0.01 pCi/m³. Plutonium-specific analyses are performed and reported if any filter from these air samplers exceeds the Rocky Flats Plant screening level of 0.01 pCi/m³ total long-lived alpha activity. Plutonium concentration data is routinely reported only for the five locations (above) which have historically produced the largest total long-lived alpha activities of the 23 onsite ambient air sampler locations. Air samplers S-02 and S-19 were inoperational during this period.

^{*} Air sampler inoperational during this period.

Table III.

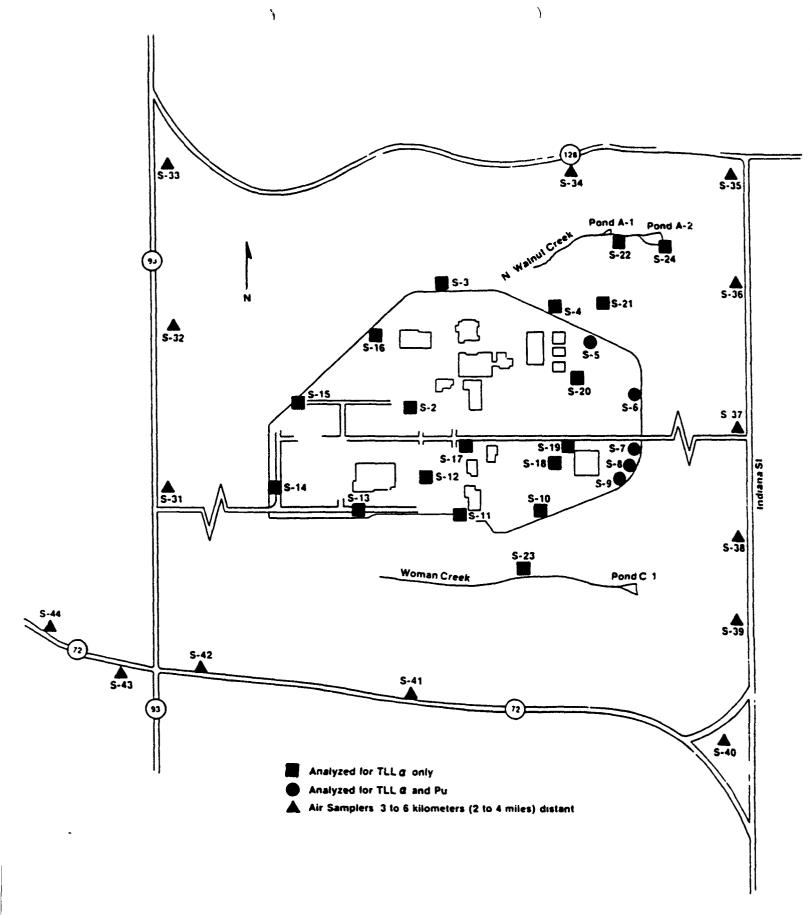
Plutonium Concentration in Ambient Air for Selected Onsite Samplers

JUNE 1989 (05/30/89 - 06/27/89)

<u>Location</u>	<u>N</u>	Volume <u>(m³)</u>	Avg. Pu Conc. (pC1/m³)	+/- Error <u>(pCi/m³)</u>
S-05	2	31000	0.000037	0.000011
S-06	2 *	33000	0.000101	0.000023
S-07	2 *	26000	0 000245	0.000064
S-08	2 *	34000	0.000277	0.000063
S-09	2 *	28000	0.000253	0.000054

NOTE. The total long-lived alpha activities of the remaining onsite ambient air sampler filters were below 0.01 pCi/m3. Plutonium-specific analyses are performed and reported if any filter from these air samplers exceeds the Rocky Flats Plant screening level of 0.01 pCi/m3 total long-lived alpha activity. Plutonium concentration data is routinely reported only for the five locations (above) which have historically produced the largest total long-lived alpha activities of the 23 onsite ambient air sampler locations.

^{*} Correction of previously reported data.

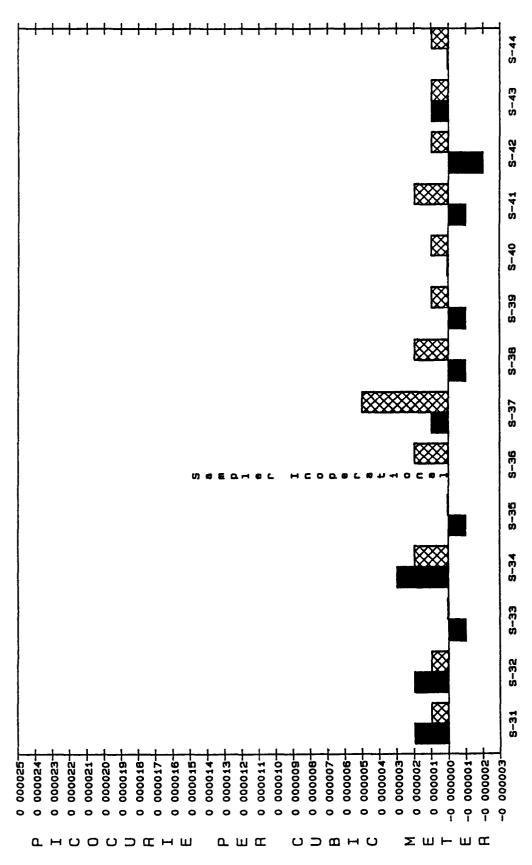


Location of Onsite and Plant Perimeter Ambient Air Samplers (Portions of figure are not to scale)

AUGUST 1989 (08/01/89 - 08/29/89)

<u>Location</u>	<u>N</u>	Volume (m³)	Pu Conc. (pC1/m³)	+/- Error <u>(pC1/m³)</u>
S-31 S-32	1	28000 31000	0.000002 0.000002	0.000003 0.000003
S-32 S-33 S-34	1	32000	-0.000001	0.000003
S-35	1 1 *	30000 27000	0.000003 -0.000001	0.000003 0.000003
S-36 S-37	1	30000	0.000001	0.00003
S-38 S-39	1 1	29000 31000	-0.000001 -0.000001	0.000003 0.000003
S-40 S-41	1 1	28000 32000	0.000000 -0.000001	0.000003 0.000003
S-42 S-43	1 1	31000 32000	-0.000002 0.000001	0.000003 0.000003
S-44	1	30000	0.000000	0.000003

^{*} Air sampler inoperational during this period.



AIR SAMPLER LOCATION

AUGUST 1989

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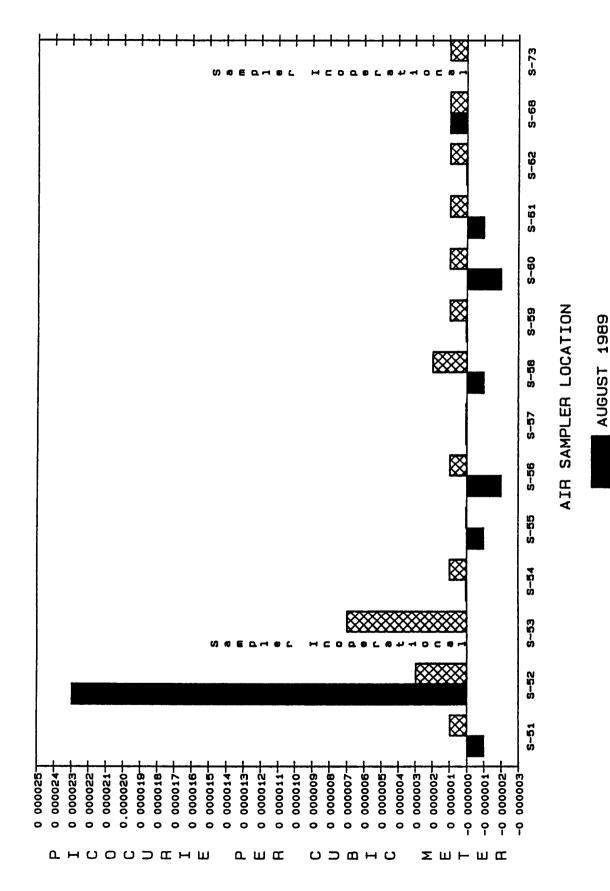
- Page 10' -

 $\label{eq:total_point} \textbf{Table V.}$ Plutonium Concentration in Ambient Air for Community Samplers

AUGUST 1989 (08/02/89 - 08/30/89)

<u>Location</u>	Community Name	<u>N</u>	Volume (m ³)	Pu Conc. <u>(pC1/m³)</u>	+/- Error (pC1/m³)
S-51	MARSHALL	1	28000	-0.000001	0.000003
S-52	JEFFCO AIRPORT	1	31000	0.000023	0.000007
S-53	SUPERIOR	*	0		
S-54	BOULDER	1	29000	0.00000	0.000003
S-55	LAFAYETTE	1	33000	-0.000001	0.000003
S-56	BROOMFIELD	1	28000	-0.000002	0.000003
S-57	WALNUT CREEK	1	29000	0.000000	0.000003
S-58	WAGNER	1	31000	-0.000001	0.000003
S-59	LEYDEN	1	32000	0.000000	0.00003
S-60	WESTMINSTER	1	28000	-0.000002	0.00003
S-61	DENVER	1	29000	-0.000001	0.000003
S-62	GOLDEN	1	30000	-0.000000	0.00003
S-68	LAKEVIEW POINTE	1	27000	0.000001	0.00003
S-73	COTTON CREEK	*	0		

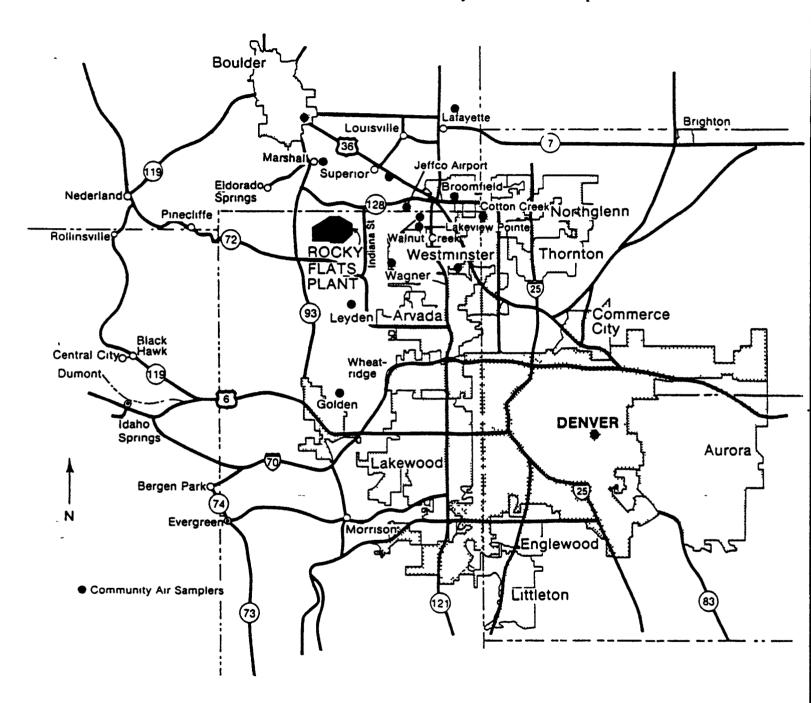
^{*} Air sampler inoperational during this period.



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Location of Community Ambient Air Samplers



18/3-

Table VI. Onsite Water Sample Results - Plutonium, Uranium, and Americium

- Holding Pond Outfall (pCi/1)

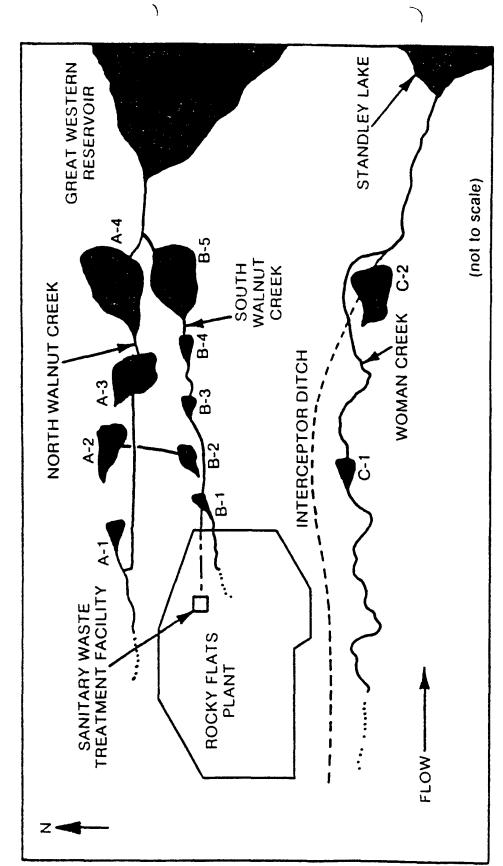
<u>Location</u>	Plutonium	<u>Uranıum</u>	Americium
Pond A-4			
8/24/89 to 8/27/89 8/28/89 to 9/1/89	0 029 ± 0.033 -0 015 ± 0.028	5.39 ± 0.33 6.36 ± 0.39	-0.003 ± 0.030 0.045 ± 0.030
Average Concentration	0.007 ± 0.031	5.88 ± 0.36	0.021 ± 0.030
Pond B-5			
8/17/89 to 8/20/89 8/21/89 to 8/25/89 8/26/89 to 8/27/89 8/28/89 to 9/1/89	0.003 ± 0.010 -0.008 ± 0.028 0.007 ± 0.031 -0.015 ± 0.028	2.00 ± 0.18 1.50 ± 0.16 1.36 ± 0.17 1.92 ± 0.18	-0.002 ± 0 009 0.024 ± 0.034 -0.018 ± 0.028 0.019 ± 0.029
Average Concentration	0.004 ± 0.026	1.70 ± 0.17	0.006 ± 0.027
Pond C-1			
No Flow			
Average Concentration			
Pond C-2			
No Discharge			
Average Concentration			
Walnut Creek at Indiana			
8/18/89 to 8/20/89 8/21/89 to 8/25/89 8/26/89 to 8/27/89 8/28/89 to 9/1/89	0.023 ± 0.013 0.002 ± 0.006 0.029 ± 0.018 0.081 ± 0.030*	3.24 ± 0.24 2.72 ± 0.20 3.79 ± 0.26 4.05 ± 0.27	0.003 ± 0.010 0.005 ± 0.006 0.024 ± 0.016 0.024 ± 0.029
Average Concentration	0.034 ± 0.019	3.45 ± 0.24	0.014 ± 0.018

^{*} Two aliquots run for this sample period.

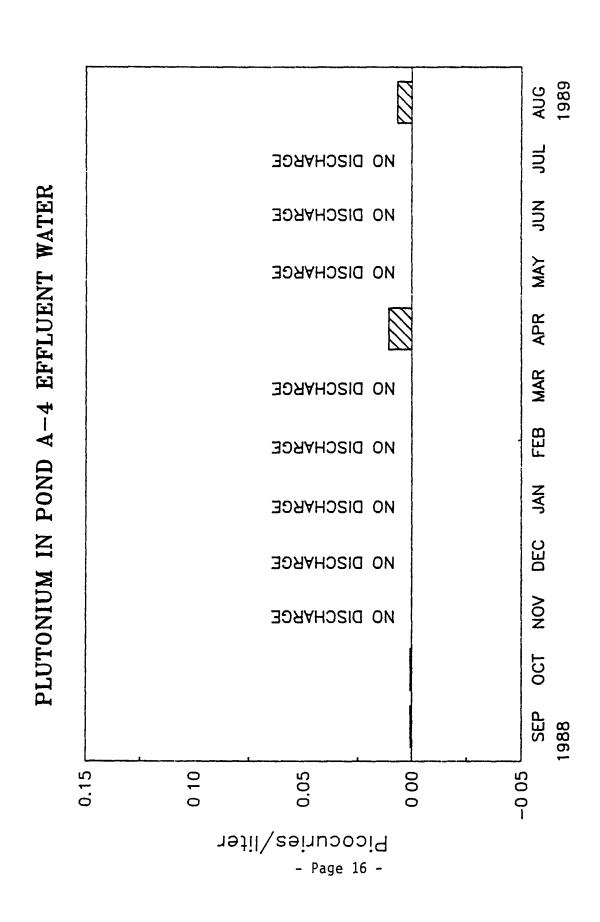
The mean of the two values will be used as discharge value for this time period in any further calculations.

^{1.} Pu239 0.170 \pm 0.030 pC1/1

² Pu239 -0.008 \pm 0.030 pC1/1



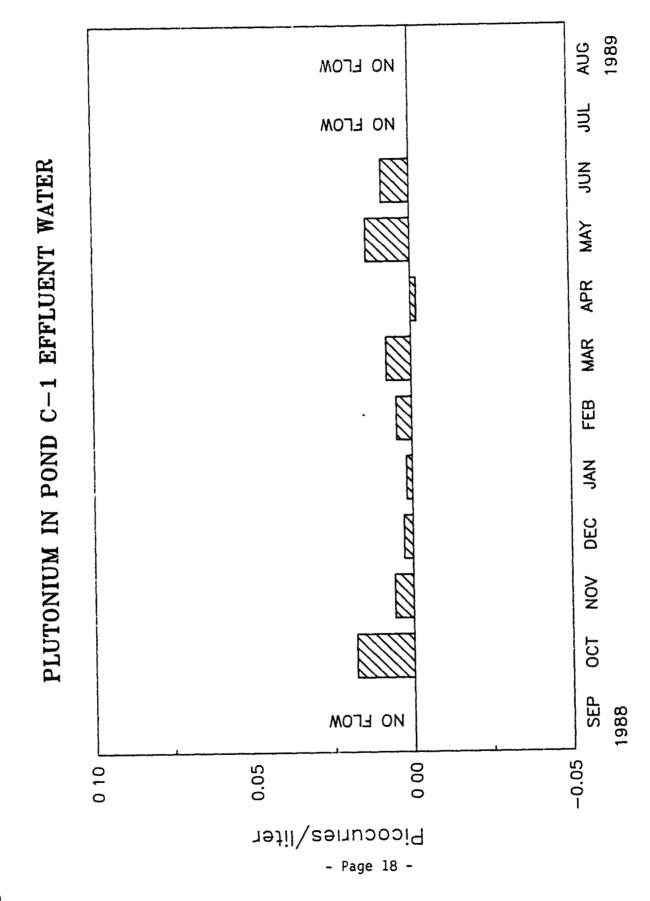
Holding Ponds and Liquid Effluent Watercourses



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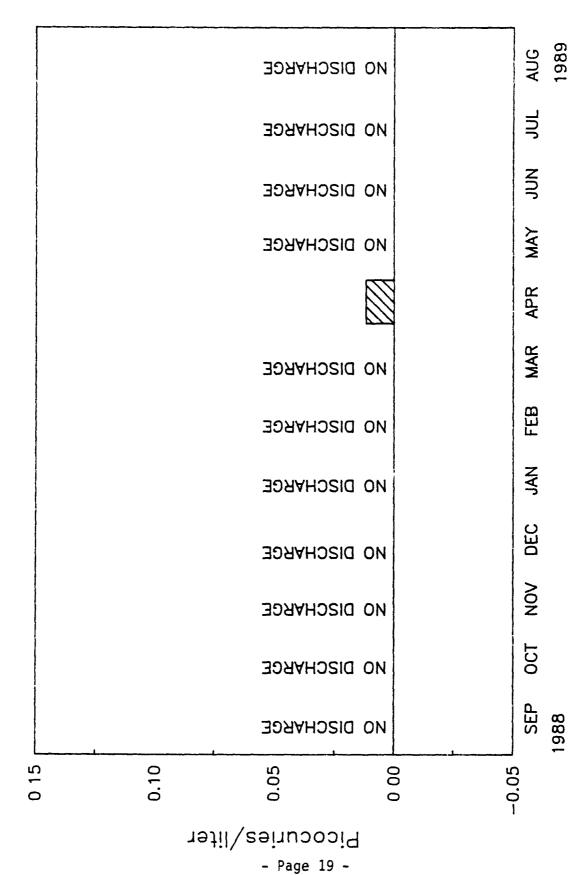
AUG 1989 JOF NO DISCHARGE PLUTONIUM IN POND B-5 EFFLUENT WATER S NO DISCHARGE MAYAPR MAR NO DISCHARGE FEB JAN NO DISCHARGE DEC **≥** OCT NO DISCHARGE SEP 1988 0 50 0.45 0.40 0 35 0 20 000 0.15 0 05 0 30 0.25 0 10 -0 05 Picocuries/liter

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PLUTONIUM IN POND C-2 EFFLUENT WATER



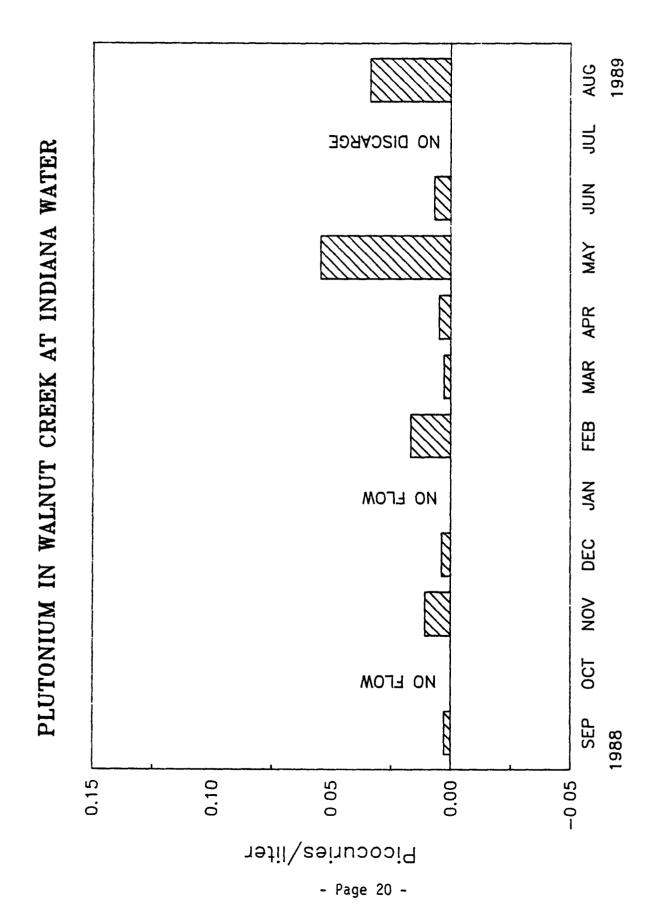


Table VII. Offsite Water Sample Results - Plutonium, Uranium, and Americium

Reservoirs (pCi/1)

Location	<u>n</u>	Plutonium	Uranıum	Americium
Great Western	1*	-0.003 ± 0.006	0.89 ± 0.15	0.002 ± 0.006
Standley Lake	1*	0.003 ± 0.006	1.05 ± 0.15	0.011 ± 0.006

Community Tap Water (pC1/1)

<u>Location</u>	<u>n</u>	Plutonium	Uranium	Americium
Boulder	1*	0.000 ± 0.006	0.00 ± 0.12	0.015 ± 0.029
Broomfield	1*	0.000 ± 0.006	0.76 ± 0.18	0.021 ± 0.007
Westminster	1*	0.002 ± 0.007	0.38 ± 0.12	0.005 ± 0.006

^{*-} Plutonium, uranium and americium analyses were performed on one sample composited from four weekly grab samples.

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Table VIII. Onsite and Offsite Water Sample Results - Tritium

<u>Tritium</u> (pCi/1)

<u>Location</u>	<u>n</u>	^c Mınımum	<u>CMaxımum</u>	^c Average
Pond A-4	8	- 10 ± 150	180 ± 110	100 ± 130
Pond B-5	15	-160 ± 140	160 ± 310	- 10 ± 230
Walnut Creek at Indiana	14	- 30 ± 310	240 ± 430	40 ± 230
Boulder	5	-210 ± 280	100 ± 280	- 40 ± 230
Broomfield	5	-150 ± 280	0 ± 100	-140 ± 230
Great Western	5	-110 ± 290	170 ± 160	10 ± 240
Standley	5	20 ± 100	260 ± 290	90 ± 240
Westminster	5	-270 ± 280	40 ± 150	-20 ± 230

Table IX. Offsite Water Sample Results - Nitrate as Nitrogen

Nitrate (as N) at Great Western Reservoir

Sample Date	Nitrate (as N) (mg/l)
08/03/89	0.04
08/10/89	<0.02
08/17/89	0.05
08/25/89	<0.02
08/31/89	0.04

Nitrate (as N) at Standley Lake

Sample Date	Nitrate (as N) (mg/l)
08/03/89	0.17
08/10/89	<0.02
08/17/89	0.02
08/25/89	0 06
08/31/89	0 02

NOTE: For some nonradioactive parameters, the concentrations that are measured at or below the minimum detectable concentration (MDC) are assigned to MDC. The less than symbol (<) indicates MDC values and calculated values that include one or more MDC's.

Table X NPDES Permit Water Sample Results

Discharge 001 (<u>Pond B-3)</u>	<u>3)</u>
No Discharge)	
_		

Parameters Biochem. Oxygen Demand, 5 Day Total Suspended Solids Nitrates as N Total Chromium Total Phosphorus Oil and Grease, Visual Total Residual Chlorine Fecal Coliforms #/	mg/l mg/l mg/l mg/l mg/l mg/l	Measured 30-Day 30-Day* Average No Discharge 10 10 0.05 8 NA NA 200	Daily m Maximum
<u>Parameter</u> pH	S.U.	MeasuredLimitsMeasurDailyDailyDailyMinimumMinimumMaximumNo Discharge6.0No Dis	Daily
Discharge 002 (Pond A-3) No Discharge			
<u>Parameters</u> Nitrates as N	mg/l	MeasuredLimitsMeasur30-Day30-Day*DailyAverageAverageMaximuNo Discharge10No Discharge	Daily
На	s.u.	MeasuredLimitsMeasuredDailyDailyDailyMinimumMinimumMaximumNo Discharge6.0No Discharge	Daily m <u>Maximum</u>
Discharge 003 (RO Pilot Plant) No Discharge			
<u>Parameter</u> pH	s.u	MeasuredLimitsMeasureDailyDailyDailyMinimumMinimumMaximumNo Discharge6.0No Discharge	Daily m Maxımum

^{*} This limitation applies when a minimum of 3 consecutive samples are taken during separate weeks

Table X NPDES Permit Water Sample Results (Continued)

Discharge 004 (RO Plan No Discharge	it)					
Parameters Total Suspended Solids Total Organic Compound Total Phosphorus Nitrates as N Total Chromium Total Residual Chlorin	is	mg/l mg/l mg/l mg/l mg/l	Measured 30-Day <u>Average</u> No Discharge	Limits 30-Day* Average 15 No 22 8 10 0.05 NA	Measured Daily Maximum Discharge	Limits Daily Maximum 25 30 12 20 0.1 0.5
Fecal Coliform		#/100 ml	7-Day <u>Average</u> No Discharge	7-Day <u>Average</u> 400	30-Day <u>Average</u> No Discha	30-Day <u>Average</u> arge 200
рН		s.u.	Daily <u>Minimum</u> No Discharge	Daily Minimum 6.0	Daily <u>Maximum</u> No Discha	Daily <u>Maximum</u> arge 9.0
Discharge 005 (Pond A-4) 9 days of discharge						
Parameters pH Nitrates as N Nonvolatile Suspended Solids	S.U. mg/1 mg/1	<u>n</u> 9 9	^C Mınımun 7.6 0.39 0	8	3.1 3.22	^c Average N/A 0.52 0.22
Discharge 006 (Pond B- 16 days of discharg						
Parameters pH Nitrates as N Nonvolatile Suspended Solids	S.U. mg/l mg/l	<u>n</u> 16 16 16	^C Mınımun 7.4 0 <0.02	8	1mum 3.0 5 0.13	^c Average N/A 0.62 <0.06
Discharge 007 (Pond C- No Discharge	· <u>2)</u>					
Parameters pH Nitrates as N Nonvolatile Suspended Solids	S.U. mg/l mg/l	<u>n</u> No Dis	^с <u>Міпіти</u> п charge	n ^c Max	<u>mum</u>	^c Average

Table XI. Water Sample Results, Nonradioactive Parameters

Walnut Creek at Indiana Street

<u>Parameters</u>		<u>n</u>	^С Мınımum	^C Maxımum	^c <u>Average</u>
pH	S.U.	Ī5	8.0	8.3	N/A
Nitrates as N	mg/l	15	<0.02	0.17	<0.08

Total Volume (gallons) = 10,490,000

Table XII.

Daily Flow Data Recorded at the Walnut Creek at Indiana Gaging Station Ponds A-4 and B-5, August, 1989

DATE	WALNUT CREEK AT INDIANA (gallons)	POND A-4 (gallons)	POND B-5** (gallons)
08/01/89	No Flow	No Discharge	No Discharge
08/02/89	H H	11 H	11 11
08/03/89	" "	17 17 14 11	II II
08/04/89	n n	" "	" " "
08/07/89	n n	n 11	" "
08/08/89	н н	" "	 H 11
08/09/89 08/10/89	11 11	 H II	 H H
08/11/89	H II	11 11	11 11
08/14/89	п н	11 11	11 11
08/15/89	15 11	11 11	11 11
08/16/89	11 11	11 11	H H
08/17/89	279,000	п н	343,000
08/18/89	654,000	11 11	707,000
08/19/89	559,000	11 11	583,000
08/20/89	559,000	H H	583,000
08/21/89	559,000	11 #	583,000
08/22/89	504,000	11 11	494,000
08/23/89	572,000	11 11	600,000
08/24/89	513,000	11 11	473,000
08/25/89	799,000	448,000	413,000
08/26/89	1,021,000	441,000	615,000
08/27/89	1,006,000	504,000	571,000
08/28/89	842,000	449,000	431,000
08/29/89	981,000	532,000	478,000
08/30/89	810,000	404,000	426,000
08/31/89	832,000	428,000	438,000
TOTAL VOLUME	10,490,000	3,206,000	7,738,000

Table XIII. Daily Flow Data Recorded at Ponds C-1 and C-2 During August, 1989

(WOMAN CREEK)

DATE	POND C-1 (gallons)	POND C-2 (gallons)
08/01/89	No Flow	No Discharge
08/02/89	11 11	16 11
08/03/89	H H	11 11
08/04/89	16 11	11 11
08/07/89	11 11	11 11
08/08/89	# #	11 11
08/09/89	11 11	rt 11
08/10/89	11 11	m H
08/11/89	H 11	11 11
08/14/89	11 11	77 11
08/15/89	89 Pf	11 11
08/16/89	If If	11 11
08/17/89	16 98	11 11
08/18/89	11 H	н н
08/21/89	PF 98	11 11
08/22/89	17 16	11 11
08/23/89	16 16	11 11
08/24/89	86 BE	n n
08/25/89	H H	H H
08/27/89	tt tt	н н
08/29/89	H 11	99 19
08/30/89	P9 18	11 11
08/31/89	98 BE	11 11
TOTAL VOLUME	No Flow	No Discharge

Appendix

RADIATION STANDARDS FOR PROTECTION OF THE PUBLIC

Introduction

The primary standards for protection of the public from radiation are based on radiation dose. Radiation dose is a means of quantifying the biological damage or risk of ionizing radiation. The unit of radiation dose is the rem or the millirem (1 rem = 1,000 mrem). Radiation protection standards for the public are annual standards, based on the projected radiation dose from a year's exposure to or intake of radioactive materials

Radiation dose is a calculated value. It is calculated by multiplying radioactivity concentrations in air and water or on contaminated surfaces by assumed intake rates (for internal exposures) or exposure times (for external exposure to penetrating radiation), then by the appropriate radiation dose conversion factors. That is:

RADIATION DOSE =
(RADIOACTIVITY CONCENTRATION) X
(INTAKE RATE/EXPOSURE TIME) X
(DOSE CONVERSION FACTOR)

The radioactivity concentrations can be determined either by measurements in the environment or by calculations using computer models. These computer models perform airborne dispersion/dose modeling of measured

building radioactivity effluents and estimated diffuse source term emissions (e.g., from resuspension from contaminated soil areas)

The assumed intake rates and dose conversion factors used are based on recommendations of national and international radiation protection advisory organizations, such as the National Council of Radiation Protection and Measurements (NCRP) and the International Commission on Radiological Protection (ICRP)

The radioactive materials of importance in calculating radiation dose to the public from Rocky Flats Plant activities include plutonium, uranium, americium, and tritium. The alpha radiation emissions from the plutonium, uranium, and americium are the primary contributors to the projected radiation dose.

Potential public radiation dose commitments, which could have resulted from Plant operations and from background (i.e., non-Plant) contributions, are calculated from average radionuclide concentrations measured at the Department of Energy (DOE) property boundary and in surrounding communities. Inhalation and water ingestion are the principal potential pathways of human exposure

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Calculation of Potential Plant Contribution to Public Radiation Dose

Pending final revision of its DOE Order for radiation protection standards for the public, DOE adopted an interim radiation protection standard for DOE environmental activities to be implemented in CY1985 (Va85). This interim standard incorporates guidance from the National Council on Radiation Protection and Measurements (NCRP), as well as the Environmental Protection Agency Clean Air Act air emission standards (as implemented in 40 CFR 61, Subpart H). Included in the interim standard is a revision of the dose

limits for members of the public Tables of radiation dose conversion factors currently used for calculating dose from intakes of radioactive materials were issued in July 1988 (US88a, US88b) The dose factors are based on the International Commission on Radiological Protection (ICRP) Publications 30 and 48 methodology and biological models for radiation dosimetry. The DOE interim standard and the dose conversion factor tables are used for assessment of any potential Rocky Flats Plant contribution to public radiation dose. The DOE radiation standards for protection of the public are given below



ICRP-, NCRP- RECOMMENDED STANDARDS FOR ALL PATHWAYS:

OCCASIONAL EXPOSURES -

500 mrem/year

EFFECTIVE DOSE EQUIVALENT*

PROLONGED EXPOSURES -

(>5 YEARS)

100 mrem/year

EFFECTIVE DOSE EQUIVALENT

INDIVIDUAL ORGAN -

5,000 mrem/year DOSE EQUIVALENT

EPA CLEAN AIR ACT STANDARDS FOR THE AIR PATHWAY ONLY:

30

WHOLE BODY -

25 mrem/year

DOSE EQUIVALENT

ANY ORGAN -

75 mrem/year DOSE EQUIVALENT

Secondary radioactivity concentration guides can be calculated from the primary radiation dose standards and used as comparison values for measured radioactivity concentrations DOE provided guidance for calculating these concentration guides - called "Derived Concentration Guides" - in a 1985 memorandum to its facilities (St85) Derived Concentration Guides (DCGs) are the concentrations which would result in an effective dose equivalent of 100 mrem from one year's chronic exposure or intake. In calculating air inhalation DCGs, DOE assumes that the exposed individual inhales 8,400 cubic meters of air at the calculated DCG during the year. Ingestion DCGs assume a water intake of 730 liters at the calculated DCG for the year The following table lists the air and water DCGs for the principal radionuclides of interest at the Rocky Flats Plant.

To determine compliance with the EPA air emissions standards, measured airborne effluent radioactivity emissions and estimated radioactivity resuspension from soil are entered into the EPA-approved atmospheric dispersion/dose calculation computer model, AIRDOS-EPA, for calculation of the maximum radiation dose that an individual in the public could receive from the air pathway only.

For comparison with the annual radiation dose standards for protection of the public, the maximum annual effective dose equivalent that a member of the public could receive as a result of Rocky Flats Plant activities is typically less than 1 mrem, or less than 1 percent of the recommended annual standard for all pathways

DOE DERIVED CONCENTRATION GUIDES FOR RADIONUCLIDES OF INTEREST AT THE ROCKY FLATS PLANT

AIR INHALATION:

Radionuclide	DCG (pCi/m³)
Pu-239, -240	0.02

WATER INGESTION:

	Radionuclide	DCG (pC1/1)
•	Pu-239, -240	30
	Am-241	30
	U-233, -234, -238	500
•	H-3	2,000,000

References

US88a DOE/EH-0070, "External Dose-Rate Conversion Factors for Calculation of Dose to the Public," U.S. Dept. of Energy, Asst. Secretary for Environment, Safety and Health, Office of Environmental Guidance and Compliance, July 1988

US88b DOE/EH-0071, "Internal Dose Conversion Factors for Calculation of Dose to the Public," U S. Dept. of Energy, Asst Secretary for Environment, Safety and Health, July 1988.

Vass Vaughan, WA, Asst Secretary, "Radiation Standards for Protection of the Public in the Vicinity of DOE Facilities," DOE memorandum from Environment, Safety and Health, August 5, 1985

Stern, R J, Director, "Preparation of Annual Site Environmental Reports for Calendar Year 1985," DOE memorandum, Office of Environmental Guidance, February 28, 1986.

*NOTE: "Dose equivalent" is a calculated value used to quantify radiation dose; it reflects the degree of biological effect from ionizing radiation. Differences in the biological effect of different types of ionizing radiation (e.g., alpha, beta, gamma, or x-rays) are accounted for in the calculation of dose equivalent

"Effective dose equivalent" is a calculated value used to allow comparisons of total health risk (based primarily on the risk of cancer mortality) from exposures of different types of ionizing radiation to different body organs. It is calculated by first calculating the dose equivalent to those organs receiving significant exposures, multiplying each organ dose equivalent by a health risk weighting factor, and then summing those products. One millirem effective dose equivalent from natural background radiation would have the same health risk as one millirem effective dose equivalent from artificially-produced sources of radiation.